

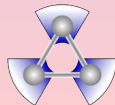
DUAL FLUID REACTOR (DFR)*

A NEW CONCEPT FOR A NUCLEAR POWER REACTOR

AHMED H. HUSSEIN

DEPARTMENT OF PHYSICS
UNIVERSITY OF NORTHERN BRITISH COLUMBIA
3333 UNIVERSITY WAY, PRINCE GEORGE, BC. CANADA
AND
INSTITUTE FOR SOLID-STATE NUCLEAR PHYSICS
LEISTIKOWSTRASSE 2, 14050 BERLIN. GERMANY

JUNE 24, 2013



*Patent pending: <http://www.google.com/patents/WO2013041085A2?cl=en>

Institution & Personel

The Dual fluid Reactor was developed at the Institute For Solid-State Nuclear Physics in Berlin, Germany. <http://festkoerper-kernphysik.de>

- Dr. Konrad Czerski,
- Dr. Armin Huke,
- Dr. Ahmed Hussein,
- Dr. Götz Ruprecht,
- Dipl.-Ing. Stephan Gottlieb, and
- Daniel Weißbach
- Dario Gigliotti.
- + supporters and advisors

World Wide Use of Nuclear Power

Some General Facts

- As of June 13, 2013, 31 countries worldwide are operating 440 nuclear reactors for electricity generation.

World Wide Use of Nuclear Power

Some General Facts

- As of June 13, 2013, 31 countries worldwide are operating 440 nuclear reactors for electricity generation.
- 69 new nuclear plants are under construction in 14 countries.

World Wide Use of Nuclear Power

Some General Facts

- As of June 13, 2013, 31 countries worldwide are operating 440 nuclear reactors for electricity generation.
- 69 new nuclear plants are under construction in 14 countries.
- Nuclear power plants provided 12.9% of the world's electricity production in 2012.

World Wide Use of Nuclear Power

Some General Facts

- As of June 13, 2013, 31 countries worldwide are operating 440 nuclear reactors for electricity generation.
- 69 new nuclear plants are under construction in 14 countries.
- Nuclear power plants provided 12.9% of the world's electricity production in 2012.
- 13 countries relied on nuclear energy to supply at least 25% of their total electricity.

World Wide Use of Nuclear Power

Current Reactors World Wide

Country	Reactors	MW _e	BkWh	% Of Total
Argentina	2	935	5,903	4.7
Armenia	1	375	2,124	26.6
Belgium	7	5,927	38,464	51.0
Brazil	2	1,884	15,170	3.1
Bulgaria	2	1,906	14,861	31.6
Canada	20	14,135	89,060	15.3
China	17	12,860	92,652	2.0
Czech RP	6	3,804	28,603	35.3
Finland	4	2,752	22,063	32.6
France	58	63,130	407,438	74.8
Germany	9	12,068	94,098	16.1
Hungary	4	1,889	14,763	45.9
India	20	4,391	29,665	3.6
Iran	1	915	1,328	0.6
Japan	50	44,215	17,230	2.1
Korea Rep.	23	20,739	143,550	30.4
Mexico	2	1,530	8,412	4.7

continued..

World Wide Use of Nuclear Power

Current Reactors World Wide

Country	Reactors	MW _e	BkWh	% Of Total
Netherlands	1	482	3,707	4.4
Pakistan	3	725	5,271	5.3
Romania	2	1,300	10,564	19.4
Russia	33	23,643	166,293	17.8
Slovakia	4	1,816	14,411	53.8
Slovenia	1	688	5,244	36.0
South Africa	2	1,860	12,398	5.1
Spain	8	7,560	58,701	20.5
Sweden	10	9,395	61,474	38.1
Switzerland	5	3,278	24,445	35.9
Taiwan, China	6	5,028	38,733	18.4
Ukraine	15	13,107	84,886	46.2
U.K.	18	9,938	63,964	18.1
U.S.	104	102,136	770,719	19.0
Total	440	374,411	2,346,194	NA

World Wide Use of Nuclear Power

Reactors Under Construction World Wide

Country	Reactors	Total MW _e
Argentina	1	692
Brazil	1	1,245
China	28	27,844
China, Taiwan	2	2,600
Finland	1	1,600
France	1	1,600
India	7	4,824
Japan	2	2,650
Pakistan	2	630
Russia	11	9,297
Slovak Republic	2	880
S. Korea	4	4,980
Ukraine	2	1,900
United Arab Emirates	2	2,690
United States	3	3,399
Total	69	66,831

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.
- High “Energy Returned On Energy Invested (EROI)”.

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.
- High “Energy Returned On Energy Invested (EROI)”.
- Can be used to provide base load energy demand. (minimum demand, constant rate).

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.
- High “Energy Returned On Energy Invested (EROI)”.
- Can be used to provide base load energy demand. (minimum demand, constant rate).
- Minimum pollution and emission of “Green House Gases (GHG)” through the life cycle.

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.
- High “Energy Returned On Energy Invested (EROI)”.
- Can be used to provide base load energy demand. (minimum demand, constant rate).
- Minimum pollution and emission of “Green House Gases (GHG)” through the life cycle.
- Long life of the resource.

Why Nuclear Power

Characteristics of a good energy source:

- High energy density.
- High “Energy Returned On Energy Invested (EROI)”.
- Can be used to provide base load energy demand. (minimum demand, constant rate).
- Minimum pollution and emission of “Green House Gases (GHG)” through the life cycle.
- Long life of the resource.
- Safe production and operation.

Why Nuclear Power

Energy Content of Various Sources

Source	Energy Density
Firewood (dry)	16 MJ/kg
Brown coal (lignite)	10 MJ/kg
Black coal (low quality)	13-23 MJ/kg
Black coal (hard)	24-30 MJ/kg
Natural Gas	38 MJ/m ³
Crude Oil	45-46 MJ/kg
Uranium - in typical reactor	500,000 MJ/kg (of natural U)

Nuclear Power

Why Nuclear Power

Energy Content of Various Sources

Source	Energy Density
Firewood (dry)	16 MJ/kg
Brown coal (lignite)	10 MJ/kg
Black coal (low quality)	13-23 MJ/kg
Black coal (hard)	24-30 MJ/kg
Natural Gas	38 MJ/m ³
Crude Oil	45-46 MJ/kg
Uranium - in typical reactor	500,000 MJ/kg (of natural U)

Energy Content of Wind and Solar

Source	Natural Energy Flows Providing 1 kW of Available Power	
Wind	Turbine Wind speed 12.5 m/s	Turbine swept area 0.85 m ²
	Wind speed 4 m/s	Turbine swept area 31.84 m ²
Solar PV	Surface perpendicular to the sun's rays at noon with sun directly overhead	Surface area 1m ²
	For an hourly average of 1kW taken over a day	Surface area 2.5 - 5 m ² depending on location

Source: http://www.mpoweruk.com/energy_resources.htm

Nuclear Power

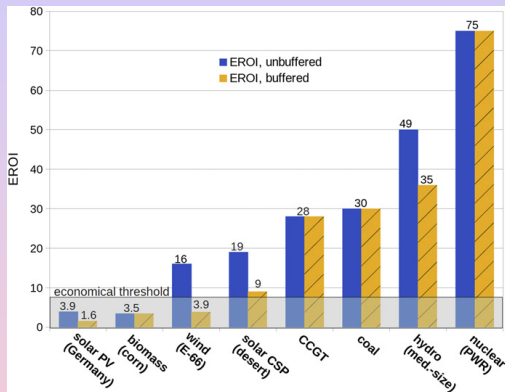


Fig. 3. EROIs of all energy techniques with economic “threshold”. *Biomass:* Maize, 55 t/ha per year harvested (wet). *Wind:* Location is Northern Schleswig Holstein (2000 full-load hours). *Coal:* Transportation not included. *Nuclear:* Enrichment 83% centrifuge, 17% diffusion. *PV:* Roof installation. *Solar CSP:* Grid connection to Europe not included.

Source:

“Energy intensities, EROIs (energy returned on invested), and energy payback times of electricity generating power plants.”
D. Weibach, G. Ruprecht, A. Huke, K. Czerski, S. Gottlieb, A. Hussein
Energy 52(2013)210-221.

Nuclear Power

Why Nuclear Power

Lifecycle Greenhouse Gas Emission Estimates
for Electricity Generators

Technology	Mean	Low	High
	g CO ₂ Eq./kWh _e		
Lignite	1,054	790	1,372
Coal	888	756	1,310
Oil	733	547	935
Natural Gas	499	362	891
Solar PV	85	13	731
Biomass	45	10	101
Nuclear	29	2	130
Hydroelectric	26	2	237
Wind	26	6	124

Source: http://www.cameco.com/common/pdf/uranium_101/Cameco_-_Corporation_Report_on_GHG_Emissions_nov_2010.pdf

Why Nuclear Power

“DeathPrint” of Various Energy Sources

Energy Source	Mortality Rate (deaths/TkWhr)	Comments
Coal global average	170,000	50% global electricity
Coal China	280,000	75% China's electricity
Coal U.S.	15,000	44% U.S. electricity
Oil	36,000	36% of energy, 8% of electricity
Natural Gas	4,000	20% global electricity
Biofuel/Biomass	24,000	21% global energy
Solar (rooftop)	440	< 1% global electricity
Wind	150	≈ 1% global electricity
Hydro global average	1,400	15% global electricity
Nuclear global average	90	17% global electricity w/Chern&Fukush

Source: <http://www.forbes.com/sites/jamesconca/2012/06/10/energys-deathprint-a-price-always-paid>

Nuclear Power

Why Nuclear Power

Life Expectancy of Fossil Fuels

Fossil Fuel	Reserves	Consumption	Year	Life Time (years)	CO ₂ Emissions (tonnes)
Natural Gas	$1.90 \times 10^{14} \text{ m}^3$	$3.37 \times 10^{12} \text{ m}^3$	2011	56	1.29×10^7
Oil	$1.47 \times 10^{12} \text{ bbl}$	$2.87 \times 10^{10} \text{ bbl}$	2011	51	1.14×10^{10}
Coal	$8.60 \times 10^{11} \text{ ton}$	$6.64 \times 10^9 \text{ ton}$	2008	129	1.30×10^{10}

Source: US Energy Information Administration
<http://www.eia.gov>

Nuclear Power

Reactors Under Construction World Wide

Reactor Type	Reactor Type Description	Number of Reactors	Total (MW _e)
BWR	Boiling Light-Water-Cooled and Moderated Reactor.	4	5,250
FBR	Fast Breeder Reactor.	2	1,259
HTGR	High-Temperature Gas-Cooled Reactor.	1	200
LWGR	Light-Water-Cooled, Graphite-Moderated Reactor.	1	915
PHWR	Pressurized Heavy-Water-Moderated and Cooled Reactor (CANDU)	5	3,212
PWR	Pressurized Light-Water-Moderated and Cooled Reactor.	56	55,995
Total		69	66,831

Sources: International Atomic Energy Agency PRIS database <http://www.iaea.org/programmes/a2/index.html>

Current Reactors

- Most commonly used reactor types for civilian power are:

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:
 - Water (light or heavy) as a coolant and moderator,

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:
 - Water (light or heavy) as a coolant and moderator,
 - High Pressures 75-160 atm.

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:
 - Water (light or heavy) as a coolant and moderator,
 - High Pressures 75-160 atm.
 - Relatively low temperatures 150 – 350°C.

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:
 - Water (light or heavy) as a coolant and moderator,
 - High Pressures 75-160 atm.
 - Relatively low temperatures 150 – 350°C.
 - Thermal neutron spectrum.

Current Reactors

- Most commonly used reactor types for civilian power are:
 - Pressurized Water Reactor (PWR),
 - Pressurized Heavy Water Reactor (Canada Deuterium Uranium CANDU),
 - Boiling Water Reactor (BWR).
- All These Reactors Use:
 - Water (light or heavy) as a coolant and moderator,
 - High Pressures 75-160 atm.
 - Relatively low temperatures 150 – 350°C.
 - Thermal neutron spectrum.
 - Solid Fuel.

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C

Problems With Current Reactors

- High Pressure \Rightarrow special materials & extensive containment.
- Low Operating Temperature \Rightarrow low heat transfer efficiency
 - PWR \Rightarrow 160 Atm, 180 – 280°C
 - CANDU \Rightarrow moderator at 1 atm coolant at 98 atm, 265 – 310°C

Problems With Current Reactors

- High Pressure \Rightarrow special materials & extensive containment.
- Low Operating Temperature \Rightarrow low heat transfer efficiency
 - PWR \Rightarrow 160 Atm, 180 – 280°C
 - CANDU \Rightarrow moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \Rightarrow 75 atm, 275 – 315°C,

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C
 - CANDU \implies moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \implies 75 atm, 275 – 315°C,
- Thermal:

Problems With Current Reactors

- High Pressure \Rightarrow special materials & extensive containment.
- Low Operating Temperature \Rightarrow low heat transfer efficiency
 - PWR \Rightarrow 160 Atm, 180 – 280°C
 - CANDU \Rightarrow moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \Rightarrow 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)

Problems With Current Reactors

- High Pressure \Rightarrow special materials & extensive containment.
- Low Operating Temperature \Rightarrow low heat transfer efficiency
 - PWR \Rightarrow 160 Atm, 180 – 280°C
 - CANDU \Rightarrow moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \Rightarrow 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)
 - Not enough to reach and maintain criticality \Rightarrow enrichment ($\approx 3 - 5\%$). Except CANDU.

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C
 - CANDU \implies moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \implies 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)
 - Not enough to reach and maintain criticality \implies enrichment ($\approx 3 - 5\%$). Except CANDU.
 - Produced actinides like Np, Pu, Am, Cm along with ^{238}U accumulate and become the long half-life component of the waste.

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C
 - CANDU \implies moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \implies 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)
 - Not enough to reach and maintain criticality \implies enrichment ($\approx 3 - 5\%$). Except CANDU.
 - Produced actinides like Np, Pu, Am, Cm along with ^{238}U accumulate and become the long half-life component of the waste.
 - Uses less than 1% of Uranium

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C
 - CANDU \implies moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \implies 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)
 - Not enough to reach and maintain criticality \implies enrichment ($\approx 3 - 5\%$). Except CANDU.
 - Produced actinides like Np, Pu, Am, Cm along with ^{238}U accumulate and become the long half-life component of the waste.
 - Uses less than 1% of Uranium
 - Requires extensive infrastructure for enrichment.

Problems With Current Reactors

- High Pressure \implies special materials & extensive containment.
- Low Operating Temperature \implies low heat transfer efficiency
 - PWR \implies 160 Atm, 180 – 280°C
 - CANDU \implies moderator at 1 atm coolant at 98 atm, 265 – 310°C
 - BWR \implies 75 atm, 275 – 315°C,
- Thermal:
 - Low neutrons per fission (average 2.5)
 - Not enough to reach and maintain criticality \implies enrichment ($\approx 3 - 5\%$). Except CANDU.
 - Produced actinides like Np, Pu, Am, Cm along with ^{238}U accumulate and become the long half-life component of the waste.
 - Uses less than 1% of Uranium
 - Requires extensive infrastructure for enrichment.
 - Accumulated Pu opens the door for weapons proliferation.

Problems With Current Reactors

- Solid Fuel

Problems With Current Reactors

- Solid Fuel
 - Requires control Rods

Problems With Current Reactors

- Solid Fuel
 - Requires control Rods
 - With the exception of CANDU, requires shutdown for refueling

Problems With Current Reactors

- Solid Fuel
 - Requires control Rods
 - With the exception of CANDU, requires shutdown for refueling
 - Requires active safety, very little passive safety features.

Alternative Reactors

- Fast Reactors:

Alternative Reactors

- Fast Reactors:
 - No Moderator \Rightarrow fast neutron spectrum.

Alternative Reactors

- Fast Reactors:
 - No Moderator \implies fast neutron spectrum.
 - Solid fuel and liquid metal cooling (Mainly Sodium, Lead)

Alternative Reactors

- Fast Reactors:
 - No Moderator \implies fast neutron spectrum.
 - Solid fuel and liquid metal cooling (Mainly Sodium, Lead)
 - Control through neutron absorbing rods

Fast Reactors

- Fast Reactors:

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .
 - Fast neutron fission produces more neutrons per fission than thermal neutron fission.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .
 - Fast neutron fission produces more neutrons per fission than thermal neutron fission.
 - After neutron losses there will be enough for breeding new fuels.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .
 - Fast neutron fission produces more neutrons per fission than thermal neutron fission.
 - After neutron losses there will be enough for breeding new fuels.
 - The high concentration of fissionable material + the extra neutron compensate for the low σ_{fission} .

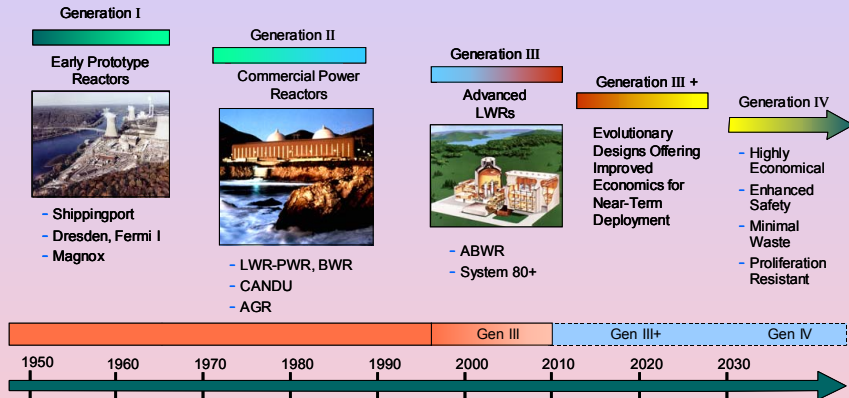
Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .
 - Fast neutron fission produces more neutrons per fission than thermal neutron fission.
 - After neutron losses there will be enough for breeding new fuels.
 - The high concentration of fissionable material + the extra neutron compensate for the low σ_{fission} .
 - $(\sigma_f/\sigma_c)_{\text{fast}} > (\sigma_f/\sigma_c)_{\text{slow}}$ for fission actinides.

Fast Reactors

- Fast Reactors:
 - Since 1951 there has been 22 fast reactors.
 - Experimental, Demo or prototype, and Commercial.
 - In China, India, France, Germany, Japan, Kazakhstan, UK, and USA.
 - All use solid fuel of Plutonium + Natural Uranium or Uranium with 20% ^{235}U .
 - Fast neutron fission produces more neutrons per fission than thermal neutron fission.
 - After neutron losses there will be enough for breeding new fuels.
 - The high concentration of fissionable material + the extra neutron compensate for the low σ_{fission} .
 - $(\sigma_f/\sigma_c)_{\text{fast}} > (\sigma_f/\sigma_c)_{\text{slow}}$ for fission actinides.
 - Proliferation resistant.

Nuclear Reactors



Source:

"A Technology Roadmap for Generation IV Nuclear Energy Systems"

U.S. DOE Nuclear Energy Research Advisory Committee and the Generation IV International Forum.

Nuclear Reactors

Generation IV Reactors

Generation IV Reactor Systems	
System Description	Acronym
Gas-Cooled Fast Reactor System	GFR
Lead-Cooled Fast Reactor System	LFR
Molten Salt Reactor System	MSR
Sodium-Cooled Fast Reactor System	SFR
Supercritical-Water-Cooled Reactor System	SCWR
Very-High-Temperature Reactor System	VHTR

Source:

"A Technology Roadmap for Generation IV Nuclear Energy Systems"

U.S. DOE Nuclear Energy Research Advisory Committee and the Generation IV International Forum.

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,
 - One with uranium enriched to 33% ²³⁵U,

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,
 - One with uranium enriched to 33% ²³⁵U,
 - In the other natural uranium was mixed wit a smaller amount ²³³U. The ²³³U was bred from thorium in another reactor.

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,
 - One with uranium enriched to 33% ²³⁵U,
 - In the other natural uranium was mixed wit a smaller amount ²³³U. The ²³³U was bred from thorium in another reactor.
- The MSRE successfully proved:

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,
 - One with uranium enriched to 33% ²³⁵U,
 - In the other natural uranium was mixed wit a smaller amount ²³³U. The ²³³U was bred from thorium in another reactor.
- The MSRE successfully proved:
 - A liquid fuel works

The Molten Salt Reactor Experiment

Some Details

- An experimental reactor at ORNL.
- Constructed 1964, went critical 1965 and was shutdown 1969.
- A 7.4 MW_{th} thermal reactor with graphite moderator.
- The fuel, which also doubled as coolant, was LiF-BeF₂-ZrF₄-UF₄ (65-29-5-1).
- Two version of the fuel/coolant were used,
 - One with uranium enriched to 33% ²³⁵U,
 - In the other natural uranium was mixed wit a smaller amount ²³³U. The ²³³U was bred from thorium in another reactor.
- The MSRE successfully proved:
 - A liquid fuel works
 - A frozen plug can provide passive safety (more on that later).

DUAL FLUID REACTOR

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.
- Liquid lead cooling was used successfully in the Soviet Alfa class submarines.

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.
- Liquid lead cooling was used successfully in the Soviet Alfa class submarines.
- DFR, however, is a fast reactor operating at 1000°C and atmospheric pressure.

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.
- Liquid lead cooling was used successfully in the Soviet Alfa class submarines.
- DFR, however, is a fast reactor operating at 1000°C and atmospheric pressure.
- DFR concept goes beyond Generation IV in using two separate fluids:

continued..

The Dual Fluid Reactor (DFR)

General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.
- Liquid lead cooling was used successfully in the Soviet Alfa class submarines.
- DFR, however, is a fast reactor operating at 1000°C and atmospheric pressure.
- DFR concept goes beyond Generation IV in using two separate fluids:
 - Molten salt for fuel, and

continued..

The Dual Fluid Reactor (DFR)

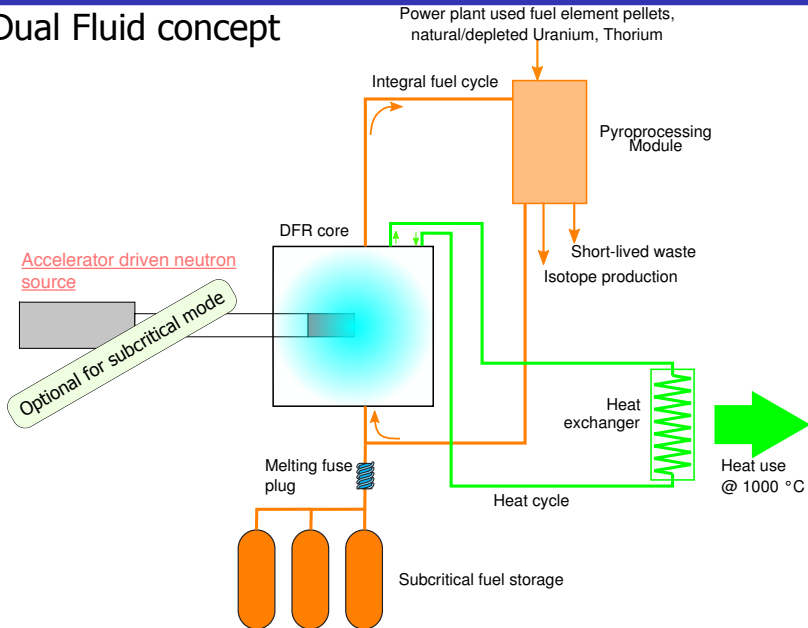
General Description

- DFR concept combines features from MSRE, LFR, and VHTR.
- All these concepts are Generation IV concepts.
- MSRE ran successfully for about four years.
- Liquid lead cooling was used successfully in the Soviet Alfa class submarines.
- DFR, however, is a fast reactor operating at 1000°C and atmospheric pressure.
- DFR concept goes beyond Generation IV in using two separate fluids:
 - Molten salt for fuel, and
 - Liquid lead for coolant.

continued..

The Dual Fluid Reactor (DFR)

The Dual Fluid concept



The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.

continued..

The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.
- Speed of fluid circulation will be optimized for maximum burn out.

continued..

The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.
- Speed of fluid circulation will be optimized for maximum burn out.
- Fuel must have melting point lower than the operating temperature, pumpable, low moderating power.

continued..

The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.
- Speed of fluid circulation will be optimized for maximum burn out.
- Fuel must have melting point lower than the operating temperature, pumpable, low moderating power.
- Two options for fuel:

continued..

The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.
- Speed of fluid circulation will be optimized for maximum burn out.
- Fuel must have melting point lower than the operating temperature, pumpable, low moderating power.
- Two options for fuel:
 - High mass halogens like UCl_3 and PuCl_3 . With ^{37}Cl to avoid neutron Capture by ^{35}Cl and forming the long lived ^{36}Cl , or

continued..

The Dual Fluid Reactor (DFR)

Fuel

- The fuel will be processed on-line using the “pyroprocessing unit”.
- Speed of fluid circulation will be optimized for maximum burn out.
- Fuel must have melting point lower than the operating temperature, pumpable, low moderating power.
- Two options for fuel:
 - High mass halogens like UCl_3 and PuCl_3 . With ^{37}Cl to avoid neutron Capture by ^{35}Cl and forming the long lived ^{36}Cl , or
 - Molten metallic fuel.

continued..

The Dual Fluid Reactor (DFR)

Fuel

- ^{239}Pu , ^{235}U , natural U, natural Th, and fissionable actinides can be used in the fuel.

The Dual Fluid Reactor (DFR)

Fuel

- ^{239}Pu , ^{235}U , natural U, natural Th, and fissionable actinides can be used in the fuel.
- DFR can consume its own generated actinides or those in the waste of other reactors.

The Dual Fluid Reactor (DFR)

Fuel

- ^{239}Pu , ^{235}U , natural U, natural Th, and fissionable actinides can be used in the fuel.
- DFR can consume its own generated actinides or those in the waste of other reactors.
- After the initial start up, depending on neutron economy and fuel mixture, DFR will produce its own fissile fuel:

The Dual Fluid Reactor (DFR)

Fuel

- ^{239}Pu , ^{235}U , natural U, natural Th, and fissionable actinides can be used in the fuel.
- DFR can consume its own generated actinides or those in the waste of other reactors.
- After the initial start up, depending on neutron economy and fuel mixture, DFR will produce its own fissile fuel:
 - ^{239}Pu from ^{238}U or

The Dual Fluid Reactor (DFR)

Fuel

- ^{239}Pu , ^{235}U , natural U, natural Th, and fissionable actinides can be used in the fuel.
- DFR can consume its own generated actinides or those in the waste of other reactors.
- After the initial start up, depending on neutron economy and fuel mixture, DFR will produce its own fissile fuel:
 - ^{239}Pu from ^{238}U or
 - ^{233}U from ^{232}Th .

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .
- Very low fast neutron capture cross-section.

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .
- Very low fast neutron capture cross-section.
- Any radioactive isotopes formed will eventually decay back to stable lead.

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .
- Very low fast neutron capture cross-section.
- Any radioactive isotopes formed will eventually decay back to stable lead.
- A good reflector for fast neutron.

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .
- Very low fast neutron capture cross-section.
- Any radioactive isotopes formed will eventually decay back to stable lead.
- A good reflector for fast neutron.
- Good heat transfer properties.

The Dual Fluid Reactor (DFR)

Coolant

- Liquid Lead
- Melting point: 327°C , Boiling point: 1749°C .
- Very low fast neutron capture cross-section.
- Any radioactive isotopes formed will eventually decay back to stable lead.
- A good reflector for fast neutron.
- Good heat transfer properties.
- Circulation speed will be optimized for heat transfer.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:
 - The fuse plug will melt and drain the liquid fuel into the subcritical tanks.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:
 - The fuse plug will melt and drain the liquid fuel into the subcritical tanks.
 - The subcritical tanks will lose the residual heat by natural convection.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:
 - The fuse plug will melt and drain the liquid fuel into the subcritical tanks.
 - The subcritical tanks will lose the residual heat by natural convection.
 - The decrease in fuel density decreases the concentration of fissile material in the fuel.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:
 - The fuse plug will melt and drain the liquid fuel into the subcritical tanks.
 - The subcritical tanks will lose the residual heat by natural convection.
 - The decrease in fuel density decreases the concentration of fissile material in the fuel.
 - The decrease in liquid lead density reduces its neutron reflectivity.

The Dual Fluid Reactor (DFR)

Safety

- Unchecked rise in core temperature due to **Loss of coolant** or any other reason, is the most serious problem in a reactor.
- DFR has a negative temperature coefficient.
- If the core temperature rises for any reason, the DFR has several inherent passive safety features:
 - The fuse plug will melt and drain the liquid fuel into the subcritical tanks.
 - The subcritical tanks will lose the residual heat by natural convection.
 - The decrease in fuel density decreases the concentration of fissile material in the fuel.
 - The decrease in liquid lead density reduces its neutron reflectivity.
 - **Doppler broadening of resonances increases the neutron capture cross-section.**

The Dual Fluid Reactor (DFR)

The Dual Fluid Reactor

www.dual-fluid-reactor.org

<http://festkoerper-kernphysik.de>



Pyrochemical
Processing Unit
(PPU)

Temporary storage.
Assorted precious metals
available at latest after
300 years (90% after 100
years)

Subcritical storage tanks for
nuclear fuel

Efficient cooling by liquid lead

Highly efficient electricity
generation at 1000 °C

Heat exchanger

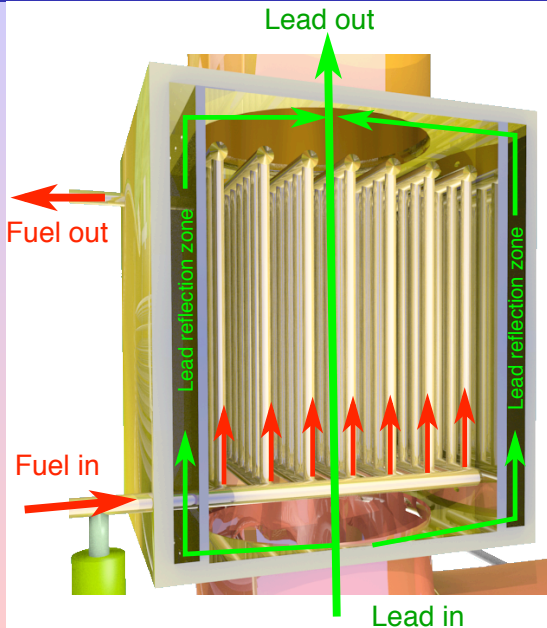
Optionally CO₂-free chemical
automotive fuel production.
Combustion in vehicles also CO₂-free.

Propeller pump

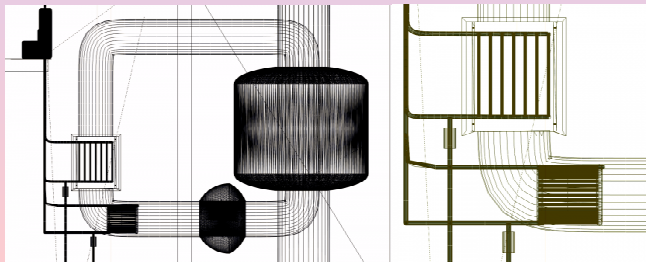
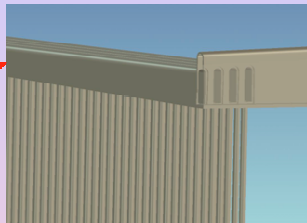
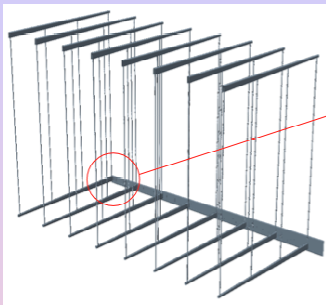
Residual heat deposited separately
from the core

IFK Berlin, June 2013

The Dual Fluid Reactor (DFR)



The Dual Fluid Reactor (DFR)



The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.
- Conversion rate is more than one (breeding mode).

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.
- Conversion rate is more than one (breeding mode).
- The neutron surplus can also be used to transmute the reactor's own fission products or those injected in from other reactors waste.

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.
- Conversion rate is more than one (breeding mode).
- The neutron surplus can also be used to transmute the reactor's own fission products or those injected in from other reactors waste.
- Self-burning mode can be achieved by consuming the neutron surplus outside the reactor core, or

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.
- Conversion rate is more than one (breeding mode).
- The neutron surplus can also be used to transmute the reactor's own fission products or those injected in from other reactors waste.
- Self-burning mode can be achieved by consuming the neutron surplus outside the reactor core, or
- By injecting Th or an inert material in the fuel.

The Dual Fluid Reactor (DFR)

Neutron Economy

- In U-Pu fuel cycle, Pu produces high neutron yield.
- Large neutron surplus remains after U-Pu conversion, which in turn produces more Pu.
- Conversion rate is more than one (breeding mode).
- The neutron surplus can also be used to transmute the reactor's own fission products or those injected in from other reactors waste.
- Self-burning mode can be achieved by consuming the neutron surplus outside the reactor core, or
- By injecting Th or an inert material in the fuel.
- Th-U fuel cycle produces much less neutron yield than U-Pu cycle.

The Dual Fluid Reactor (DFR)

Reactors: DFR and water-moderated

DFR

- Consumes “waste”
- Consumes natural Uranium and natural Thorium
- Produces fuel for water-moderated reactors, if desired
- Produces CO₂-neutral chemical fuels
- Has a closed internal fuel cycle

vs.

Water-moderated Reactor

- Produces “waste”
- Consumes U-235
- Can not produce chemical fuels
- Low power density, high costs
- Relies on fuel cycle infrastructure

Should not be an “either/or”

DFRs could be deployed on the site of water-moderated reactors, complementing their operation and closing their fuel cycle

Economics of DFR

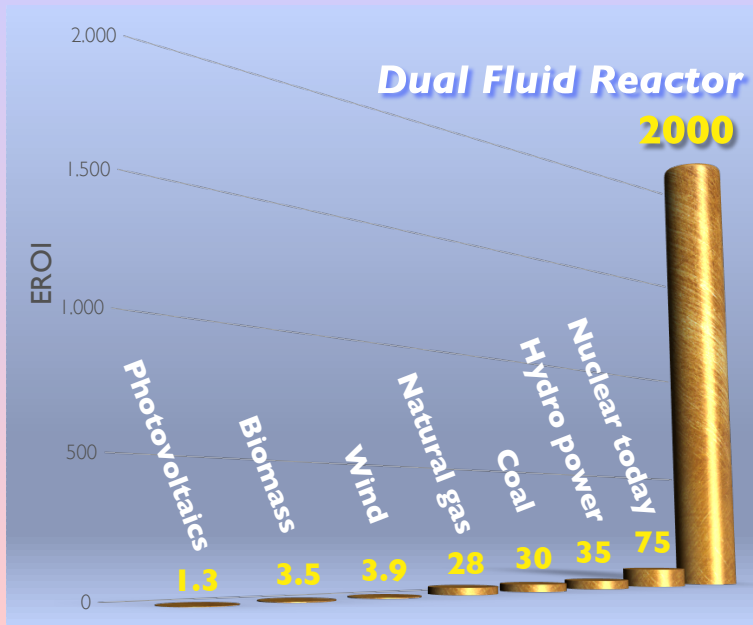
Item	Energy input of the DFR		
	Units or Total (1000 kg)	Energy Inventory (TJ/1000 kg)	Total Inventory (TJ)
Concrete containment for reactor, fission products and turbine building	21000	0.0014	30
High performance refractory metals and ceramics (PPU and core)	60	0.5	30
High temperature isolation material for PPU and core	100	0.1	10
Initial load, isotopically purified ^{37}Cl + fuel	25 + 60	2.5/0.4	50 + 25
Refractory metals and ceramics for the heat exchanger	180	0.5	90
Isolation and structural materials, heat exchanger	300	0.1	30
Unfabricated, low-alloyed metal for fission product encapsulation	3000	0.033	100
Structural materials (steel) for non-nuclear part	1000	0.02	20
Lead coolant	1200	0.036	45

continued..

Economics of DFR

Item	Energy input of the DFR		
	Units or Total (1000 kg)	Energy Inventory (TJ/1000 kg)	Total Inventory (TJ)
Turbines with generators	3	40	120
Mechanical engineering parts			150
Cooling tower (special concrete)	20000	0.003	60
Refueling, 1200 kg/a actinides over 50 years	≈ 60	0.4	≈ 25
^{37}Cl loss compensation	2	2.5	5
Maintenance, high-performance refractories+ isolation for 1 new core	30 + 50	0.5/0.1	20
Maintenance, 50% of other reactor parts, refractories+ isolation	90 + 175	0.5/0.1	62.5
Maintenance, 50% of mechanical engineering and turbines			135
Maintenance electricity, 2MW over 20 days/a and heating, 50*0.2 TJ			182.5
Sum			1190
Output over 50 years lifetime, ≈ 1500 MW net, ≈ 8300 full-load hours			2,250,000

The Dual Fluid Reactor (DFR)



Estimated Construction Costs (Million US\$)

Item	500 MW _e DFR	1500 MW _e DFR
Concrete containment for reactor, earthquake-proof	100	130
Reactor with primary circuit, features including a facility for pyrochemistry	250	300
Secondary gas loop	60	150
Supercritical water turbine 500 MWe (3x), generator, transformer	250	580
Tertiary cooling system with cooling tower	140	250
Additional bunkers	100	200
Planning and building authority, contingency	130	200
Sum	1000	1800
Costs per installed power	2 US\$/W	1.2 US\$/W

continued..

Construction Costs

- The above costs are well below the 3.3 US\$/W_e for a modern nuclear power station like the third generation of the European Pressurized Water Reactor.

Construction Costs

- The above costs are well below the 3.3 US\$/W_e for a modern nuclear power station like the third generation of the European Pressurized Water Reactor.
- The above estimates use a more expensive external cooling system. A different choice could reduce the costs by 10%.

Operational Costs

Estimated Annual DFR Operating Costs (million US\$)

Item	500 MW _e	1500 MW _e
Operating personnel: 30 man-years (3 shifts 10/12 man-years each 130,000 US\$)	4	5
Operating supplies	1.5	2.5
Nuclear fuel: 500 kg (330 US\$ mining, 330 US\$ transport, 650 US\$ per kg waste management)	0.5	1.5
Maintenance, conventional section (2.5% building costs per annum)	9.5	25
Maintenance, nuclear and pyrochemical section (2% building costs per annum)	5	7
Reserve for dismantling (25% of the building cost of 1000/1900 million US\$)	5	9
Administration, safety	2.5	4
Sum	26	54

Electricity Costs

Estimated Total Costs OF Electricity Produced With DFR In US¢/kWh

Item	500 MW _e	1500 MW _e
Capital costs	0.50	0.30
Operating costs	0.65	0.45
Sum	1.15	0.75
DFR/PWR or Coal	0.30	0.20

Note1: PWR \equiv Pressurized Water Reactor, most commonly used reactor.

Note2: Cost of electricity produced by Coal fired power stations are close to those produced by PWRs.

continued..

Applications of DFR

Application of DFR

- Production of Electricity,

Applications of DFR

Application of DFR

- Production of Electricity,
- Water desalination,

Applications of DFR

Application of DFR

- Production of Electricity,
- Water desalination,
- Production of Synthetic Automotive Fuels for transportation,

Applications of DFR

Application of DFR

- Production of Electricity,
- Water desalination,
- Production of Synthetic Automotive Fuels for transportation,
- Production of medical & industrial isotopes.

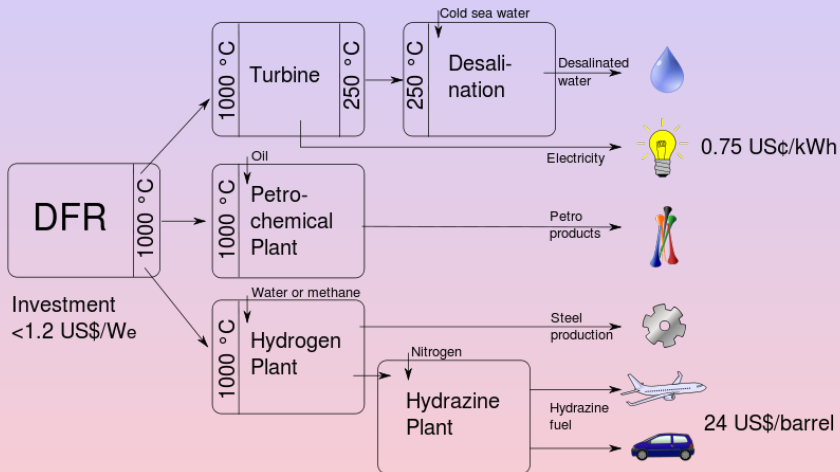
Applications of DFR

Application of DFR

- Production of Electricity,
- Water desalination,
- Production of Synthetic Automotive Fuels for transportation,
- Production of medical & industrial isotopes.
- Many others.

continued..

DFR Applications



Synthetic Automotive Fuels

- DFR operates at a very high temperature of 1000°C , which provides a very efficient use of heat.

continued..

Synthetic Automotive Fuels

- DFR operates at a very high temperature of 1000°C , which provides a very efficient use of heat.
- This combined with low construction and operational costs reduces the cost per unit heat.

continued..

Synthetic Automotive Fuels

- DFR operates at a very high temperature of 1000°C , which provides a very efficient use of heat.
- This combined with low construction and operational costs reduces the cost per unit heat.
- DFR can then be used as a very low cost source of Hydrogen gas from water through combined electrolysis and thermal decomposition

continued..

Synthetic Automotive Fuels

- DFR operates at a very high temperature of 1000°C , which provides a very efficient use of heat.
- This combined with low construction and operational costs reduces the cost per unit heat.
- DFR can then be used as a very low cost source of Hydrogen gas from water through combined electrolysis and thermal decomposition
- This cheap Hydrogen makes the production of Nitrogen and Silicon based synthetic automotive fuels economically viable.

continued..

Synthetic Automotive Fuels

- The most commonly used automotive fuels are Gasoline, Diesel, Natural Gas and Liquified Petroleum Gas. All these fuels (except Natural Gas) are extracts from crude oil.

continued..

Synthetic Automotive Fuels

- The most commonly used automotive fuels are Gasoline, Diesel, Natural Gas and Liquified Petroleum Gas. All these fuels (except Natural Gas) are extracts from crude oil.
- In addition some automotive fuels can be synthesized from other sources like biomass, corn, natural gas, etc.

continued..

Synthetic Automotive Fuels

- The most commonly used automotive fuels are Gasoline, Diesel, Natural Gas and Liquified Petroleum Gas. All these fuels (except Natural Gas) are extracts from crude oil.
- In addition some automotive fuels can be synthesized from other sources like biomass, corn, natural gas, etc.
- All these fuels are “organic fuels”, based on carbon and hydrogen. As a result, they all produce Carbon Dioxide as they burn.

continued..

Synthetic Automotive Fuels

- There are other synthesized fuels that are **not** based on Carbon. Examples are **Hydrazine** and **Silane**

continued..

Synthetic Automotive Fuels

- There are other synthesized fuels that are **not** based on Carbon. Examples are **Hydrazine** and **Silane**
- Hydrazine is made of **Nitrogen** and Hydrogen while Silane is made of **Silicon** and Hydrogen

continued..

Synthetic Automotive Fuels

- There are other synthesized fuels that are **not** based on Carbon. Examples are **Hydrazine** and **Silane**
- Hydrazine is made of **Nitrogen** and Hydrogen while Silane is made of **Silicon** and Hydrogen
- **Hydrazine** has been used as rocket fuel in NASA's space program.

continued..

Synthetic Automotive Fuels

- There are other synthesized fuels that are **not** based on Carbon. Examples are **Hydrazine** and **Silane**
- Hydrazine is made of **Nitrogen** and Hydrogen while Silane is made of **Silicon** and Hydrogen
- Hydrazine has been used as rocket fuel in NASA's space program.
- It can be used in automotive combustion engines with proper additives, resulting in **Water Vapour** and **Nitrogen** as waste.

continued..

Synthetic Automotive Fuels

- There are other synthesized fuels that are **not** based on Carbon. Examples are **Hydrazine** and **Silane**
- Hydrazine is made of **Nitrogen** and Hydrogen while Silane is made of **Silicon** and Hydrogen
- Hydrazine has been used as rocket fuel in NASA's space program.
- It can be used in automotive combustion engines with proper additives, resulting in **Water Vapour** and **Nitrogen** as waste.
- Similarly, Silane burns to **Water Vapour** and **Silicon Nitride**, an inert compound.

continued..

Synthetic Automotive Fuels

- In addition, Hydrazine and Silane have a wide range of industrial applications.

continued..

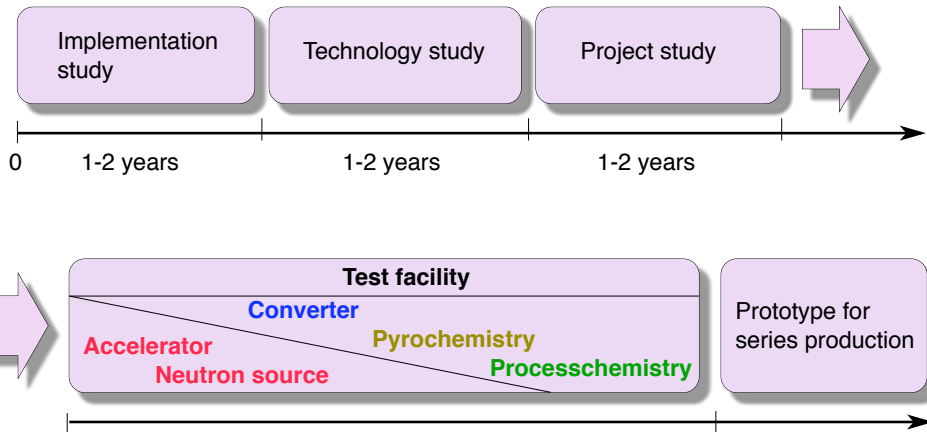
Synthetic Automotive Fuels

- In addition, Hydrazine and Silane have a wide range of industrial applications.
- The hazardous properties and toxicity of these fuels are similar to Gasoline.

continued..

Overall Plan

Estimated timeline



Investments Required

Phase I

- Phase I will last for 2-3 years.

Investments Required

Phase I

- Phase I will last for 2-3 years.
- Requires \approx \$2.25-3.00 million of investment. This money will be used for personnel and equipment:

Investments Required

Phase I

- Phase I will last for 2-3 years.
- Requires \approx \$2.25-3.00 million of investment. This money will be used for personnel and equipment:
 - Computer hardware, software, and/or computing time @ \approx \$50,000.

Investments Required

Phase I

- Phase I will last for 2-3 years.
- Requires \approx \$2.25-3.00 million of investment. This money will be used for personnel and equipment:
 - Computer hardware, software, and/or computing time @ \approx \$50,000.
 - About 6 scientists and engineers for design and simulation work @ \approx \$1.2 million, includes salaries and benefits, conference travel and miscellaneous.

Investments Required

Phase I

- Phase I will last for 2-3 years.
- Requires \approx \$2.25-3.00 million of investment. This money will be used for personnel and equipment:
 - Computer hardware, software, and/or computing time @ \approx \$50,000.
 - About 6 scientists and engineers for design and simulation work @ \approx \$1.2 million, includes salaries and benefits, conference travel and miscellaneous.
 - 1.5 scientists for experimental work on material testing and pyroprocessing component testing @ \approx \$1.2 million, includes salaries and benefits, equipment and laboratory space rental.

Possible Support

Possible Support

- Canadian Center for Nuclear Innovation (CCNI), Saskatchewan.

Possible Support

Possible Support

- Canadian Center for Nuclear Innovation (CCNI), Saskatchewan.
- Government of Dubai, United Arab Emirates.

Possible Support

Possible Support

- Canadian Center for Nuclear Innovation (CCNI), Saskatchewan.
- Government of Dubai, United Arab Emirates.
- Government of Egypt: Ministry of Higher Education and Nuclear Power Plant Authority.

Possible Support

Possible Support

- Canadian Center for Nuclear Innovation (CCNI), Saskatchewan.
- Government of Dubai, United Arab Emirates.
- Government of Egypt: Ministry of Higher Education and Nuclear Power Plant Authority.
- European Commission for Energy, Euratom, and Sustainable Nuclear Energy Technology Platform.

Possible Support

Possible Support

- Canadian Center for Nuclear Innovation (CCNI), Saskatchewan.
- Government of Dubai, United Arab Emirates.
- Government of Egypt: Ministry of Higher Education and Nuclear Power Plant Authority.
- European Commission for Energy, Euratom, and Sustainable Nuclear Energy Technology Platform.
- TRIUMF????